





Results of a Dosimetric System for Personal Dose Equivalent Assessment



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1. INTRODUCTION

Since the introduction of thermoluminescence (TL) for dosimetric purposes, a lot of efforts have been made to explain and also to improve its application for personal, clinical and environmental dosimetry. The use of thermoluminescent dosemeter is attractive because of its small size, low cost and they also can assess the radiation doses to demonstrate compliance with regulations on dose limits. CaSO₄: Dy is one of the most useful and sensitive thermoluminescence material for radiation dosimetry. Because of its high sensitivity to gamma radiation and stability of response, it is often used to assess the radiation dose to the general public from nuclear or radiological facilities [1,2,3] and also for personal dosimetry. The Individual Monitoring Service of the Instituto de Pesquisas Energéticas e Nucleares (IPEN/CNEN-SP) monitors approximately 1500 workers on a monthly basis, from 14 facilities spread all over the country, using thermoluminesce dosemeters (TLD). This paper summarises the obtained results with the intercomparison that was done with two different dosimetric systems for evaluate the personal dose equivalent .

- -To analyse the results of the dosimetric system as a part of a radiation monitoring programme of occupationally exposed workers;
- -To compare the assessment of personal dose equivalent Hp(10) according to national and international recommendations [4].

3. MATERIALS AND METHODS

TL Radiation Detectors and Badge

- CaSO_{4:} Dy sintered discs: Thickness = 0.8 mm; Diameter = 6.0 mm from IPEN 137 Cs gamma source: STS OB85/1 Irradiator 128 GBq (09/2011)
- Plastic badges with different metal filters Figures 1 (Type 1) and 2 (Type 2) X Radiation Quality N-60: Pantak-Seifert, Model ISOVOLT 160 HS
 - ISO Water Filled Phamton (30 x 30 x 15) cm

Readout Equipment

• Harshaw 3500 Automatic TLD Reader

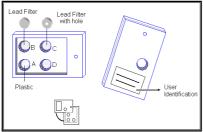


Figure 1. Dosimeter used for personal monitoring at Ipen (Type 1).

Identification

Irradiation Set-up

Figure 2. Dosimeter developed at IPEN using copper and lead filters (Type 2).

4. RESULTS

Reproducibility

- Dosimeters: CaSO₄:Dy TLD (Types 1 and 2)
- · Reading cycles: 10
- Standard annealing: 300°C/1h for CaSO₄:Dy;
- · Absorbed dose: 5mGy
- Reproducibility: better than 4.5% for both dosemeters type

Dose Response Curves

- Absorbed dose range: 0.2 to 10 mSv
- Radiation quality: 137Cs and N60 X rays beams
- · Irradiation conditions: ISO Water Filled Phantom
- Calibration curves for gamma radiation:
- CaSO₄:Dy TLD Type 1 Figure 3 and CaSO₄:Dy TLD Type 1 Figure 4

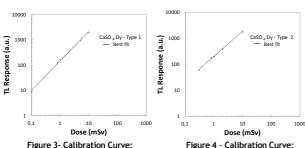


Figure 3- Calibration Curve: CaSO₄:Dy type 1 CaSO₄:Dy type 2

Table 1 - Personal dose equivalent results of the intercomparison.

Radiation Quality	Conventional True H _p (10) _T (mSv)	CaSO ₄ :Dy(Type 1)		CaSO₄:Dy (Type 2)	
		Mean Evaluated H _p (10) _E (mSv)	Deviation H _p (10) _E	Mean Evaluated H _n (10) _F	Deviation H _p (10) _E
		(11154)	Ӊ ₍ 10) _T	(mSv)	Ӊ _₀ (10) _Т
N60	0.3	0.31±0.01	1.03	0.31±0.02	1.03
	2.0	2.17±0.26	1.09	2.19±0.15	1.10
	10.0	10.09±0.06	1.01	10.00±0.13	1.00
¹³⁷ Cs	0.2	0.21±0.01	1.05	0.23±0.04	1.15
	2.0	2.12±0.24	1.06	2.25±0.18	1.13
	10.0	10.22±0.26	1.02	10.06±0.40	1.01

 CaSO₄:Dv dosimeters: the deviation between evaluated dose and conventional true dose were less than 10% for X radiation. For ¹³⁷Cs gamma rays, the deviation had larger values for CaSO₄:Dy type 2 dosimeters. In this case, probably there was a difference in the irradiation positioning.

5. CONCLUSIONS

- The dosimetric systems studied are able to measure the operational quantity personal dose equivalent Hp(10) according to ICRU for normal incidence.
- The differences between evaluated and conventional true dose were in the range of 1% to 15%, that are inside national requirements,
- To improve the personal dose equivalent evaluation technique, periodic intercomparison is needed.

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- 4. International Commission on Radiation Units and Measurement. Determination of Dose Equivalents Resulting from External Radiation Sources Part 2. Report 43 (Bethesda, MD: ICRU Publications), 1988.